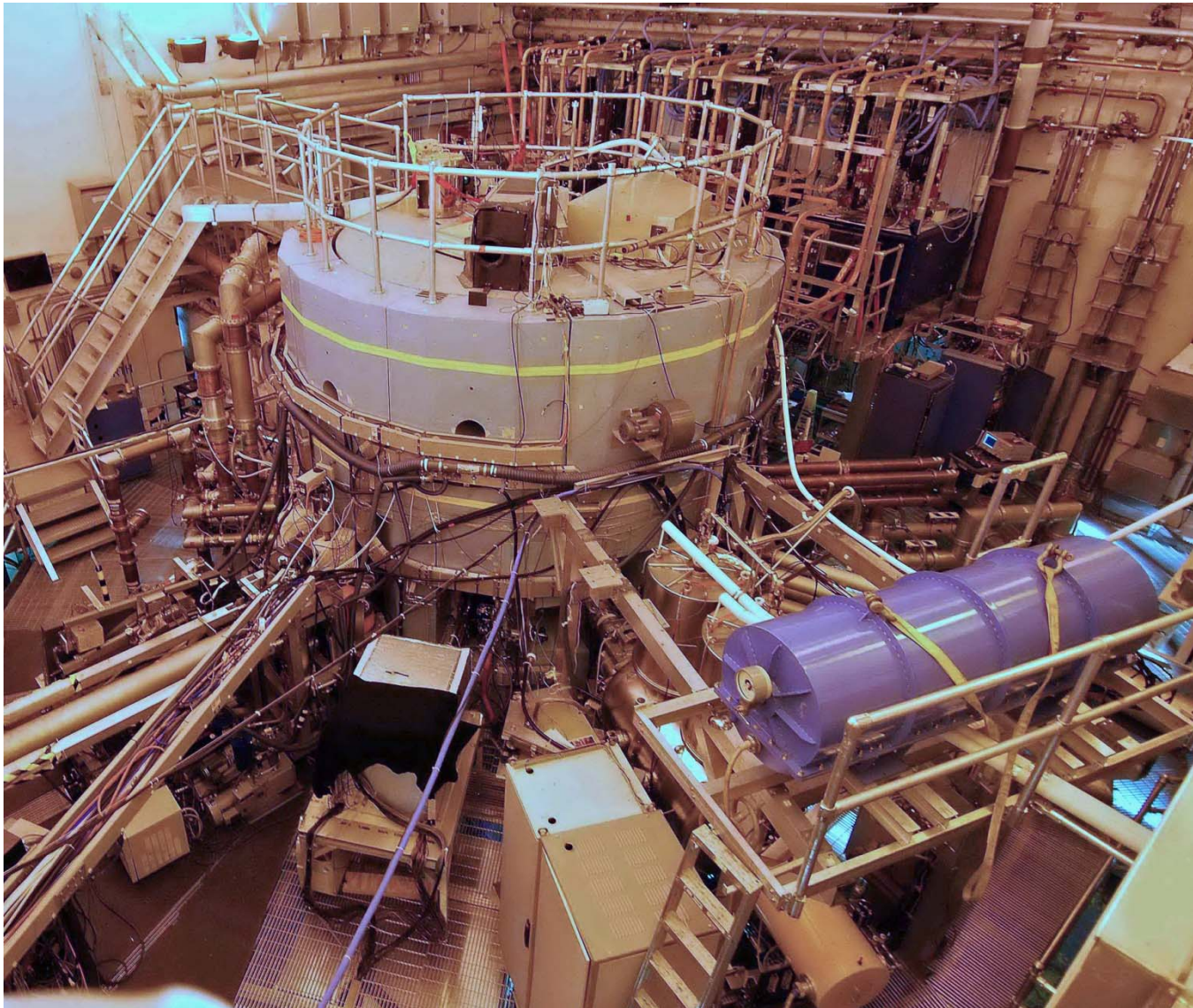


Alcator C-Mod Research Highlights and Plans



Presented by E. Marmor
on behalf of the Alcator
Team
FPA Annual Meeting
2 December 2009

*Developing the steady
state, high-Z wall,
high-field tokamak for
ITER and beyond*

C-Mod research program focuses on areas of unique capability, ITER relevance



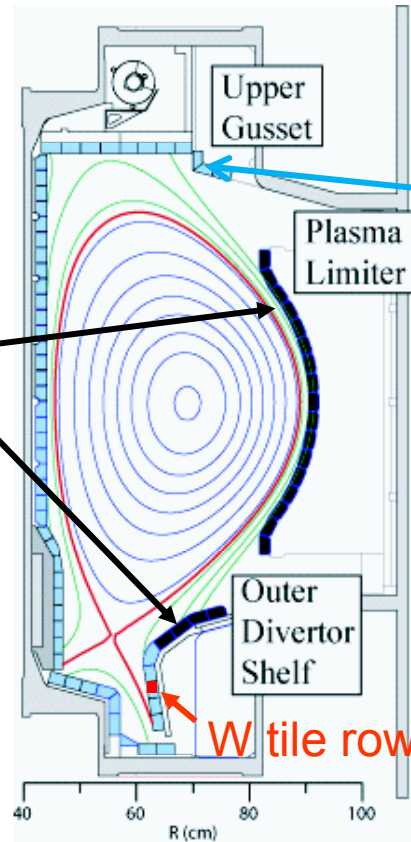
- **C-Mod restarted operation in June 2009 after a successful inspection of both machine and alternator.**
 - Many new diagnostics.
- **Broad science campaign, with particular emphasis on ITER needs and requests.**
- **Experiments exploit key C-Mod features, eg.**
 - Solid metal walls; Mo, W: D retention and recovery
 - High divertor heat fluxes: Power handling, impurity generation.
 - High density and neutral opacity: Pedestals and n_e control.
 - ICRF and LHCD at ITER B_T , density: H&CD physics
 - High pressure ($\langle P \rangle$ up to 1.8 atm): Disruption mitigation
- **Graduate students are key, integral members of the scientific team**
 - **Currently 32 full time doing their PhD research on C-Mod**
- **Strong emphasis on comparison of detailed measurements with simulations.**

Recent Research Highlights

- Many new and interesting results from recent research operations
 - Hydrogenic retention
 - Improved confinement “I-mode”
 - Neon and nitrogen seeded plasmas (all regimes)
 - H-mode pedestal physics
 - Disruption mitigation
 - B-coated Mo tile operation
 - ITER discharge development
 - Edge turbulence
 - Gyro-kinetic modeling of core turbulence measurements
 - Modeling of Lower Hybrid coupling and propagation
 - SOL transport, divertor heat flux (FY10 joint research milestone)

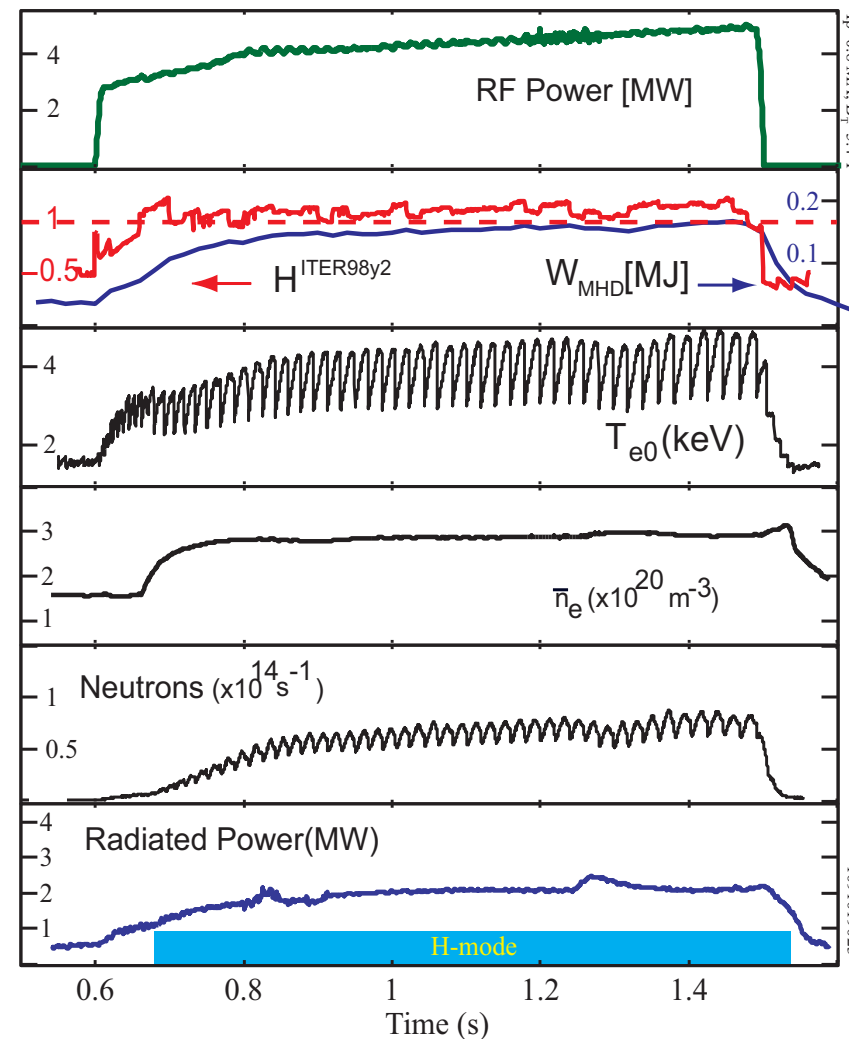
Progress in handling high input power

- Impurity generation with RF heating, metal walls is a long-standing issue.
- **During torus opening, coated selected PFC tile areas with B to better understand RF erosion.**
- Reduced Mo influx $\sim 10X$, extended the RF energy input between boronizations (to ~ 100 MJ).



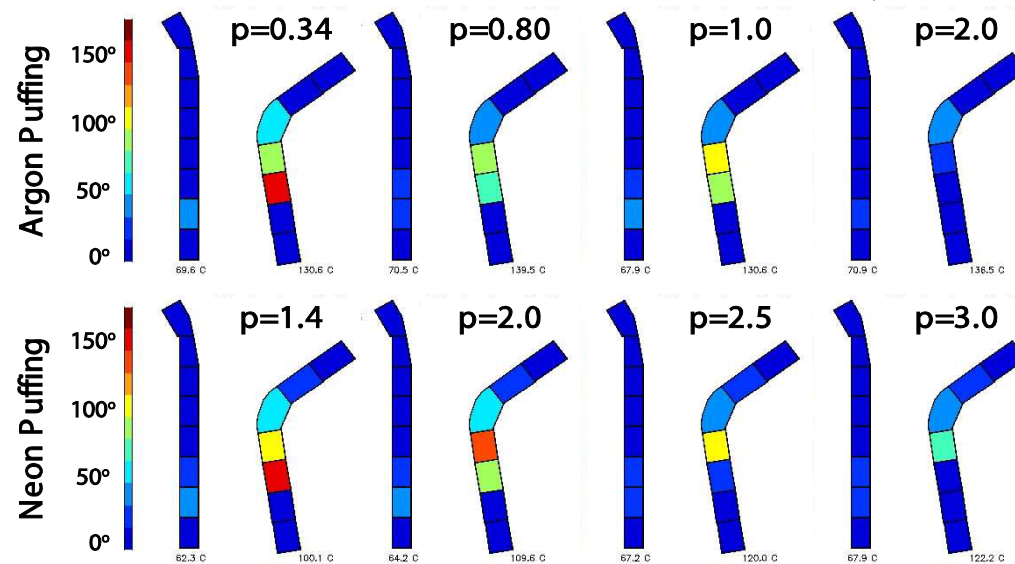
- Many experiments this campaign used high power (4-6 MW) ICRF.
- At these high heat fluxes, divertor tile heating becomes an issue.
- **Impurity seeding (Neon or N) was found to reduce T_{div} , prevent high Z injections, while maintaining good H-mode confinement** (as is hoped on ITER).

Moly



Seeding reduced divertor tile heating, and impurity injections.

Divertor temperature rise



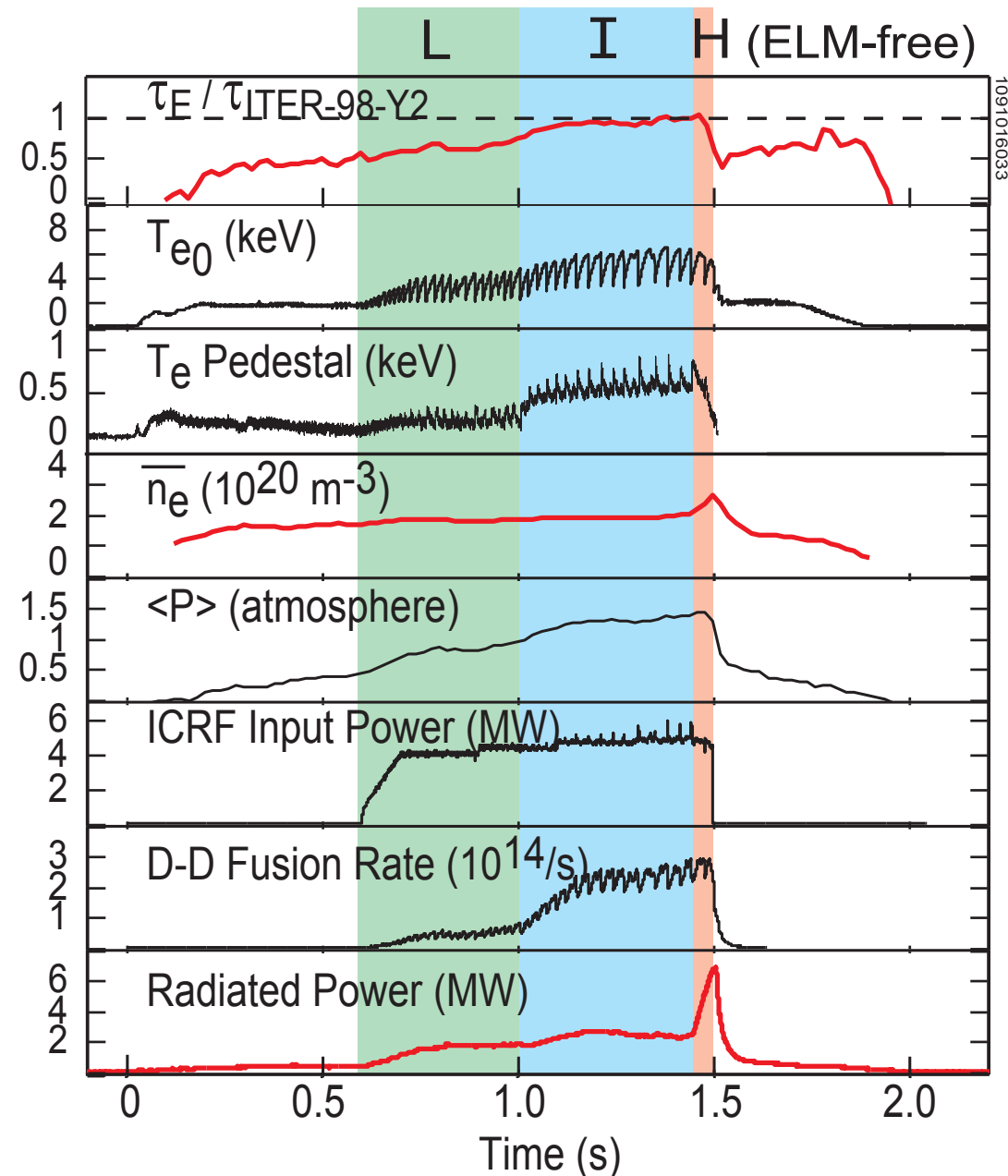
increased seeding →

- With either Argon or Neon, seeding greatly reduces peak divertor tile temperatures.
- Also eliminates high Z impurity injections, which can be a problem due to extremely high power density on C-Mod. This reduces ICRF trips.
- Net result can be higher performance H-modes with Neon seeding – starting to use in other high power experiments.

I-mode: H-mode Energy Confinement, L-mode Particle Confinement

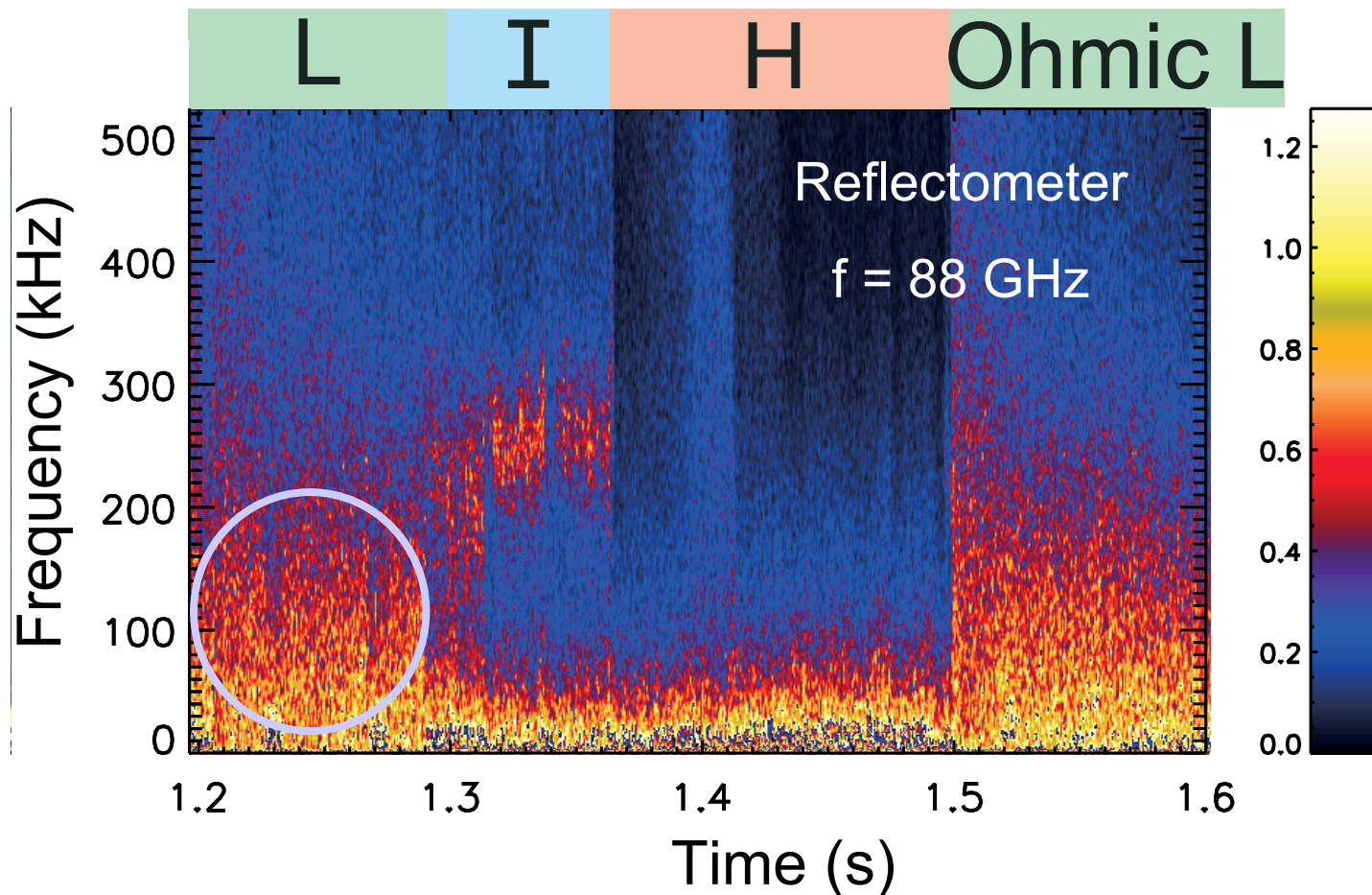
- Obtained with unfavorable drift (high L-H thresholds)*
- Globally, the regime is characterized by high energy confinement, often matching H-mode scaling ($H_{98y2} \sim 1.$)
- But, no particle barrier or impurity accumulation
 - With cryopumping, density can be controlled at the level of the ohmic target
 - P_{rad} stays very low.
- **Does not require recent boronization**
- Compatible with low Z impurity seeding

$B=5.6 \text{ T}, I_p=1.2 \text{ MA}, q_{95}=3.3$



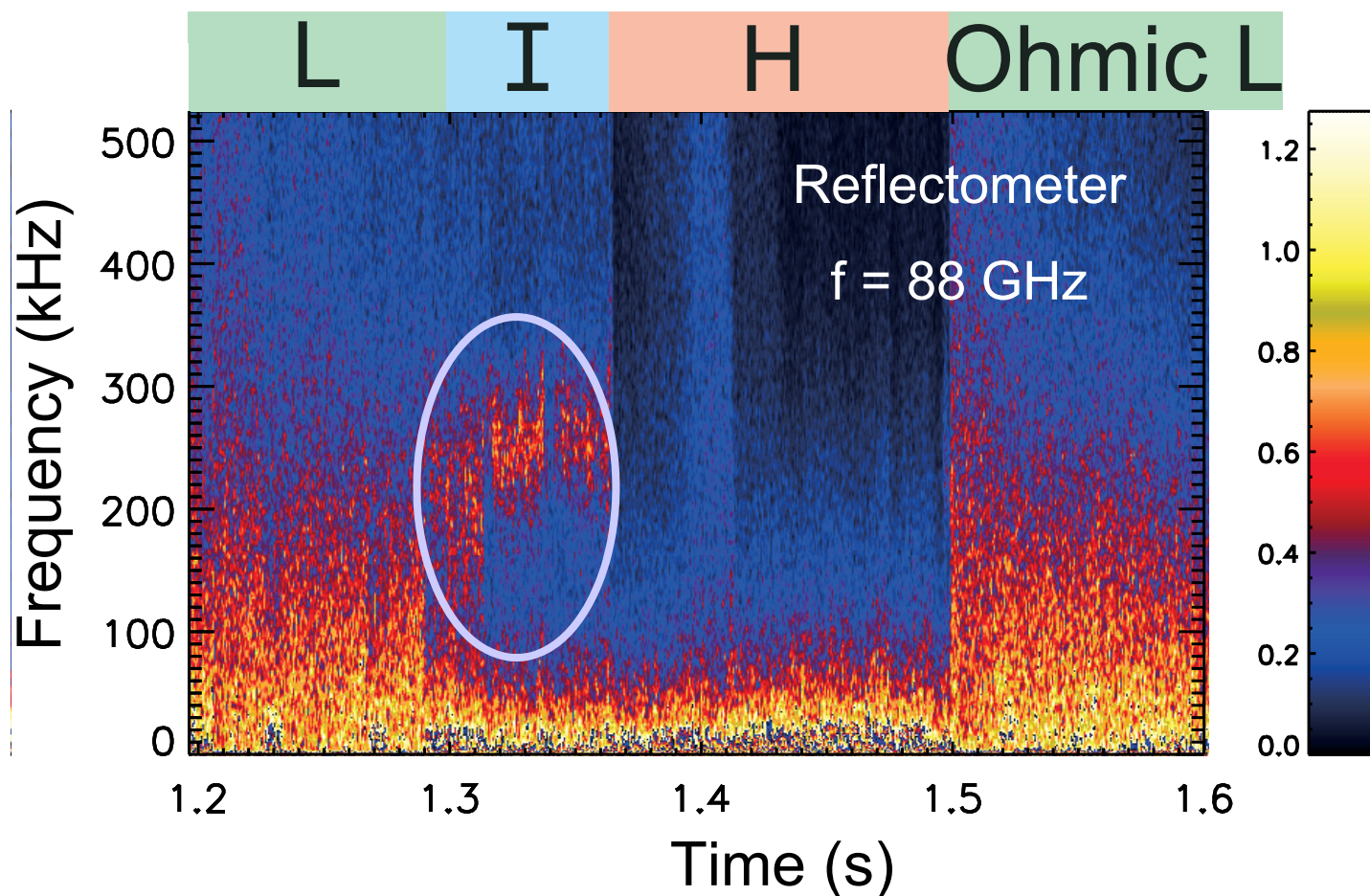
*See Ryter, et al., PPCF **40**(1998)725.

Edge/Pedestal Density and Magnetics Fluctuations in L-, I- and H-mode



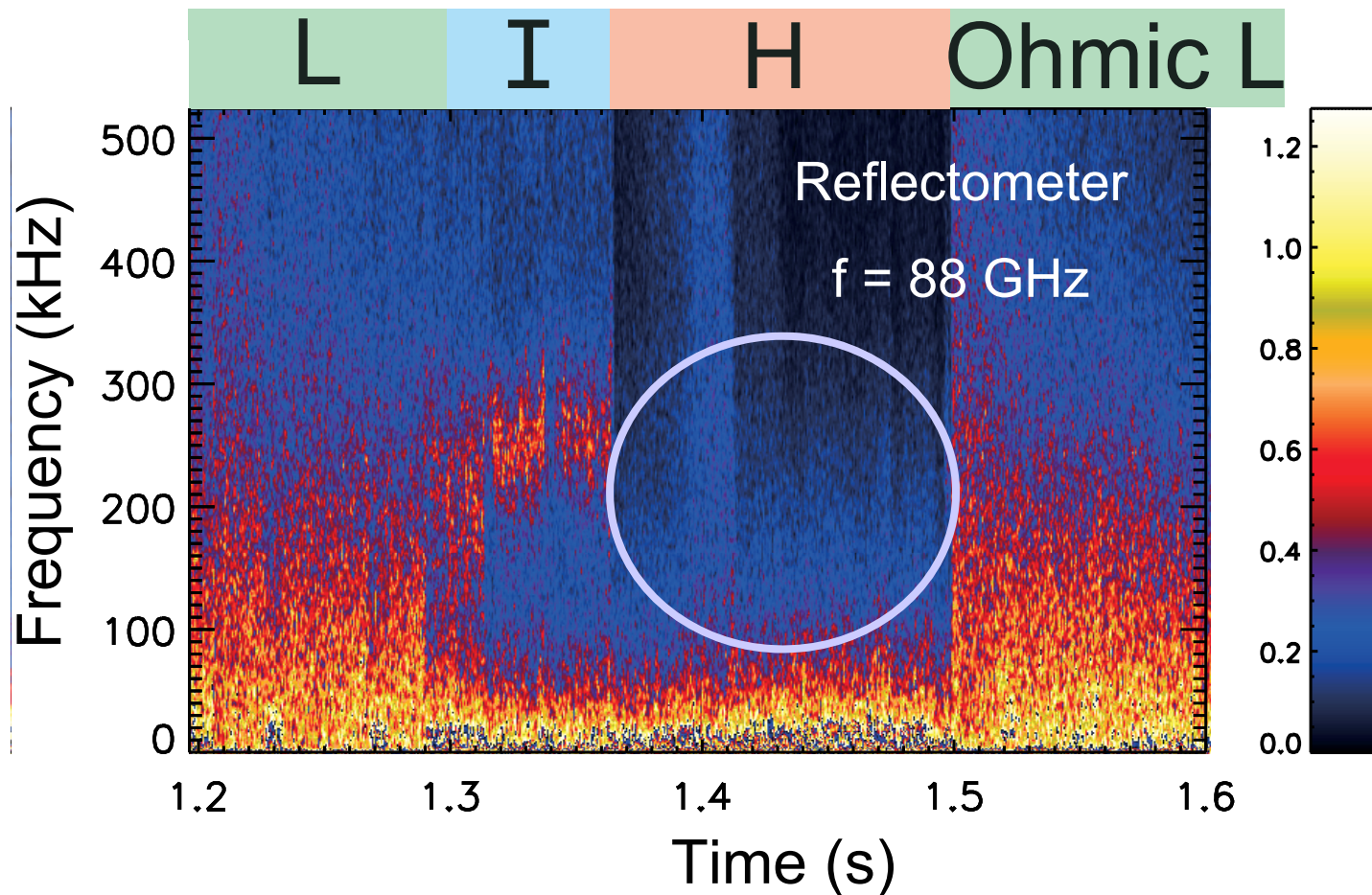
- L-mode: broadband fluctuations (50 – 200 kHz) drive energy and particle transport
- I-mode: broad band reduced, ~200 kHz appears; particle transport similar to L, energy transport suppressed
- ELM-free H-mode: ~200 kHz also gone, impurity accumulation

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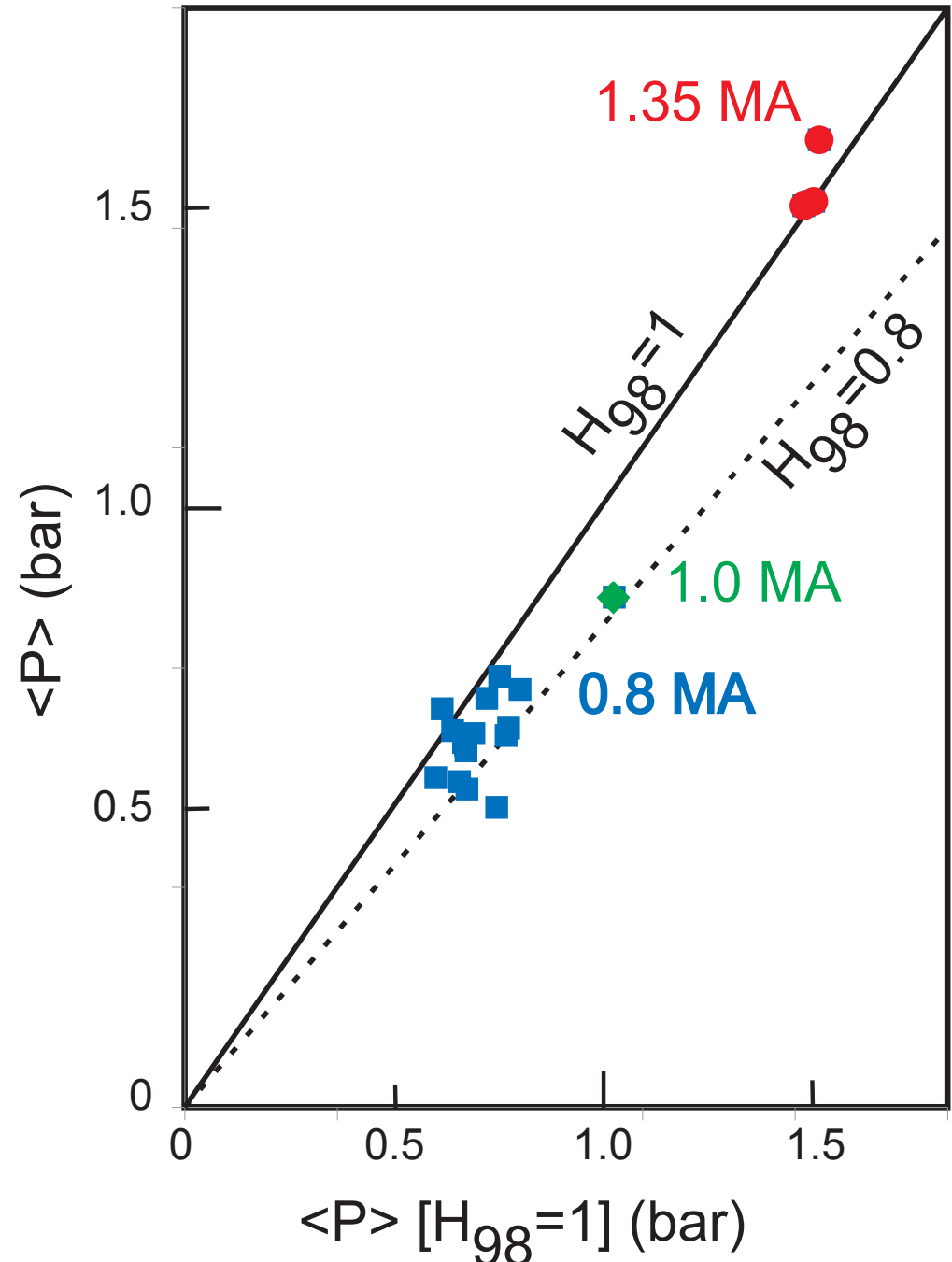
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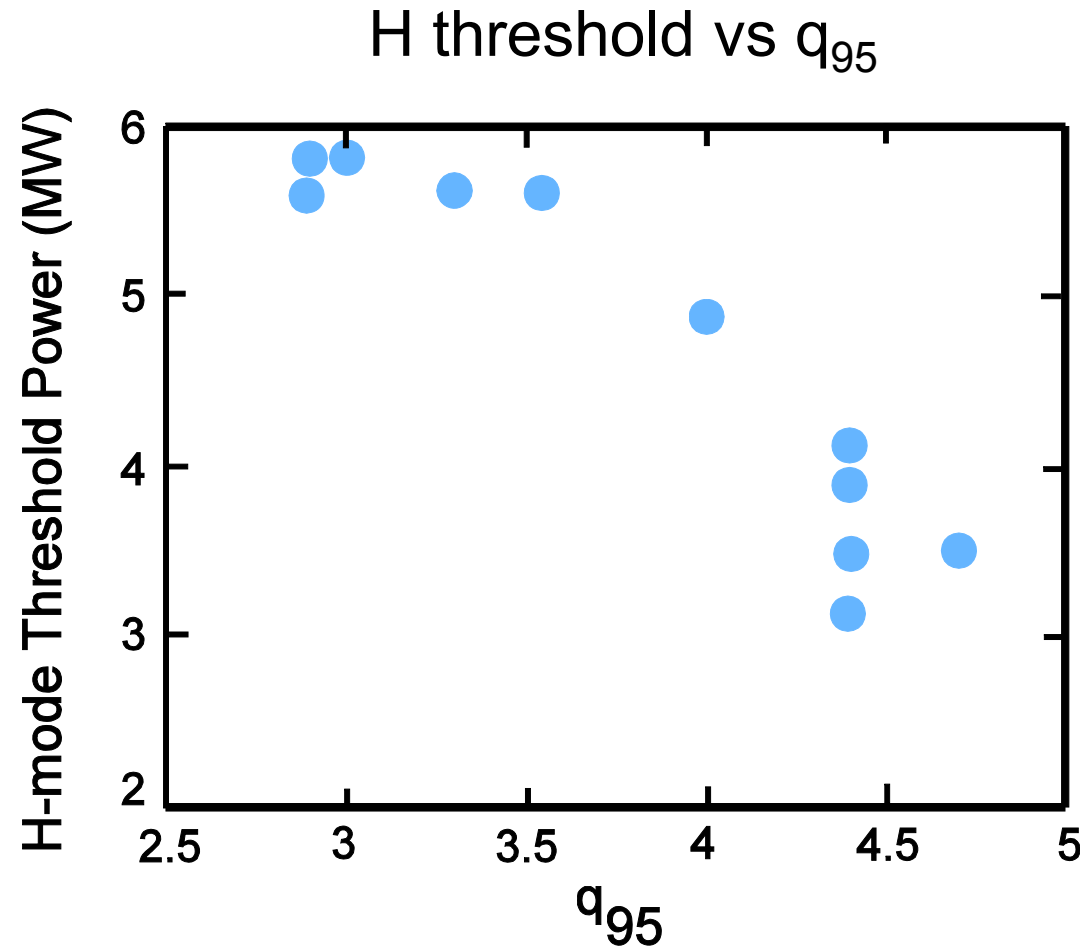
Parameter scans revealing operational space for I-mode

- Confinement quality improves at highest pressures (power, current)
 - But need to stay out of H-mode (density barrier)
- With unfavorable drift, H-mode threshold appears highest with a combination of
 - Low q_{95} (<3.5)
 - Strong shaping
- ELMs not required for edge particle regulation
- Interest in exploring application to ITER
 - alternate regime in case of difficulties with H-mode or ELM control



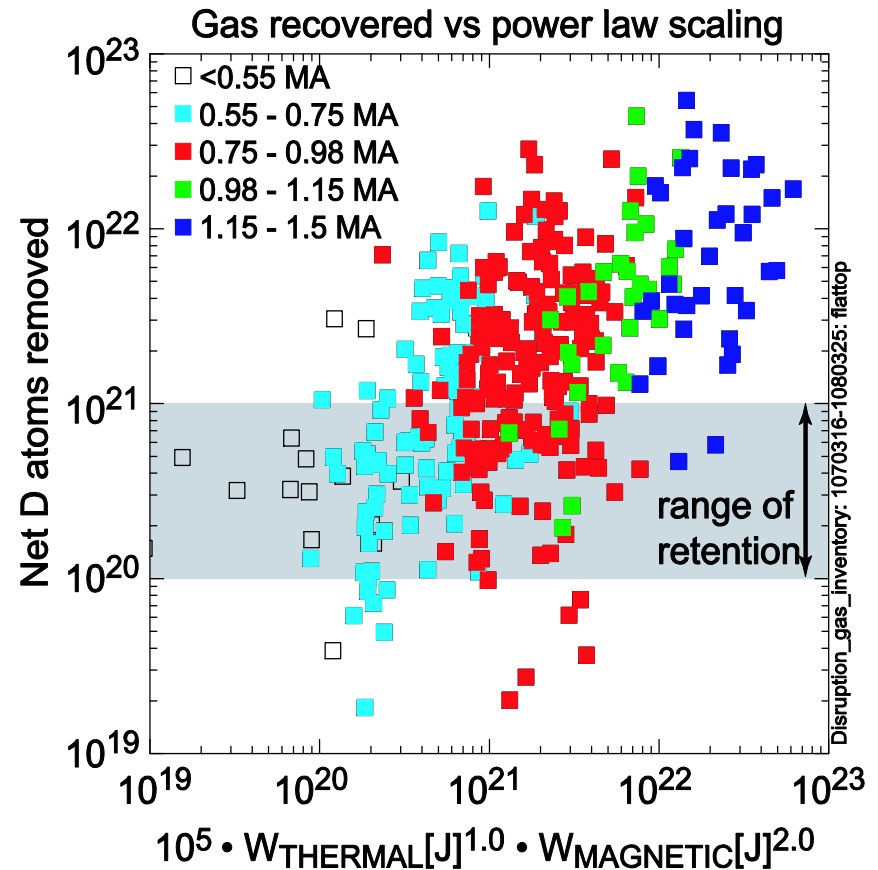
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 - But need to stay out of H-mode (density barrier)
- With unfavorable drift, H-mode threshold appears highest with a combination of
 - Low q_{95} (<3.5)
 - Strong shaping
 - As high as 4x the ITER scaling for threshold with normal drift
- ELMs not required for edge particle regulation
- Interest in exploring application to ITER
 - alternate regime in case of difficulties with H-mode or ELM control



Significant recovery of D from metal walls following disruptions

- Past C-Mod experiments showed unexpectedly high retention of D in Mo, W PFCs
 - 1-2% in single discharges, comparable to C.
 - But, campaign-integrated retention is $\sim 1000\times$ lower.
- Large release of retained gas following disruptions.
 - Scaling with magnetic energy ($\sim W_{\text{MAG}}^2$) is stronger than with thermal energy ($\sim W_{\text{TH}}$).
- Results are consistent with ‘flash heating’ of key surfaces, underscoring the crucial role of wall temperature in retention.

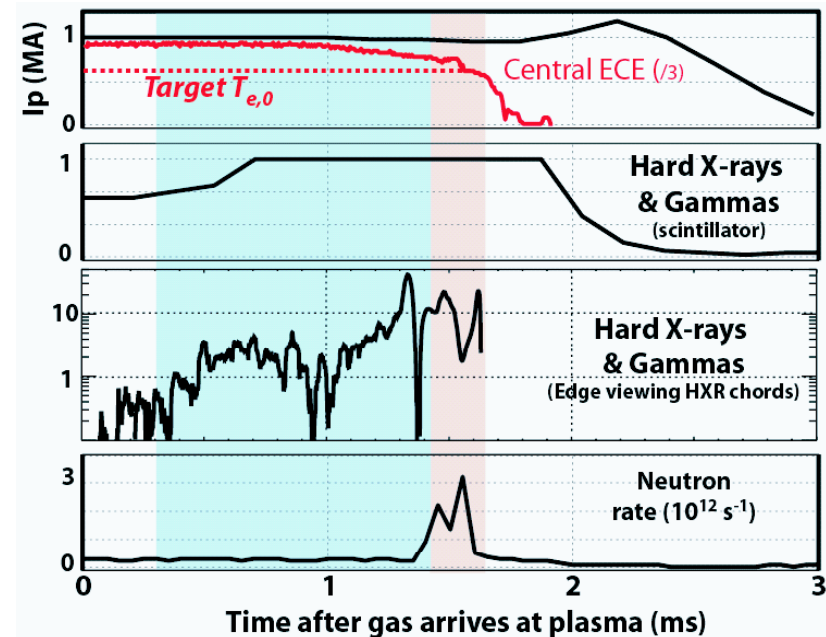


Scaling of C-Mod results to ITER suggests that reduced energy (~ 20 MJ) disruptions could release substantial T.

Rapid loss of runaway electrons during gas-jet-mitigated disruptions

- Massive Gas Injection disruption mitigation is optimized by He/Ar mixtures.
- Experiments with fast electron seeding by LHCD show rapid loss of runaway electrons during the thermal quench.
- NIMROD modeling (Val Izzo) shows stochastic fields cause rapid loss.

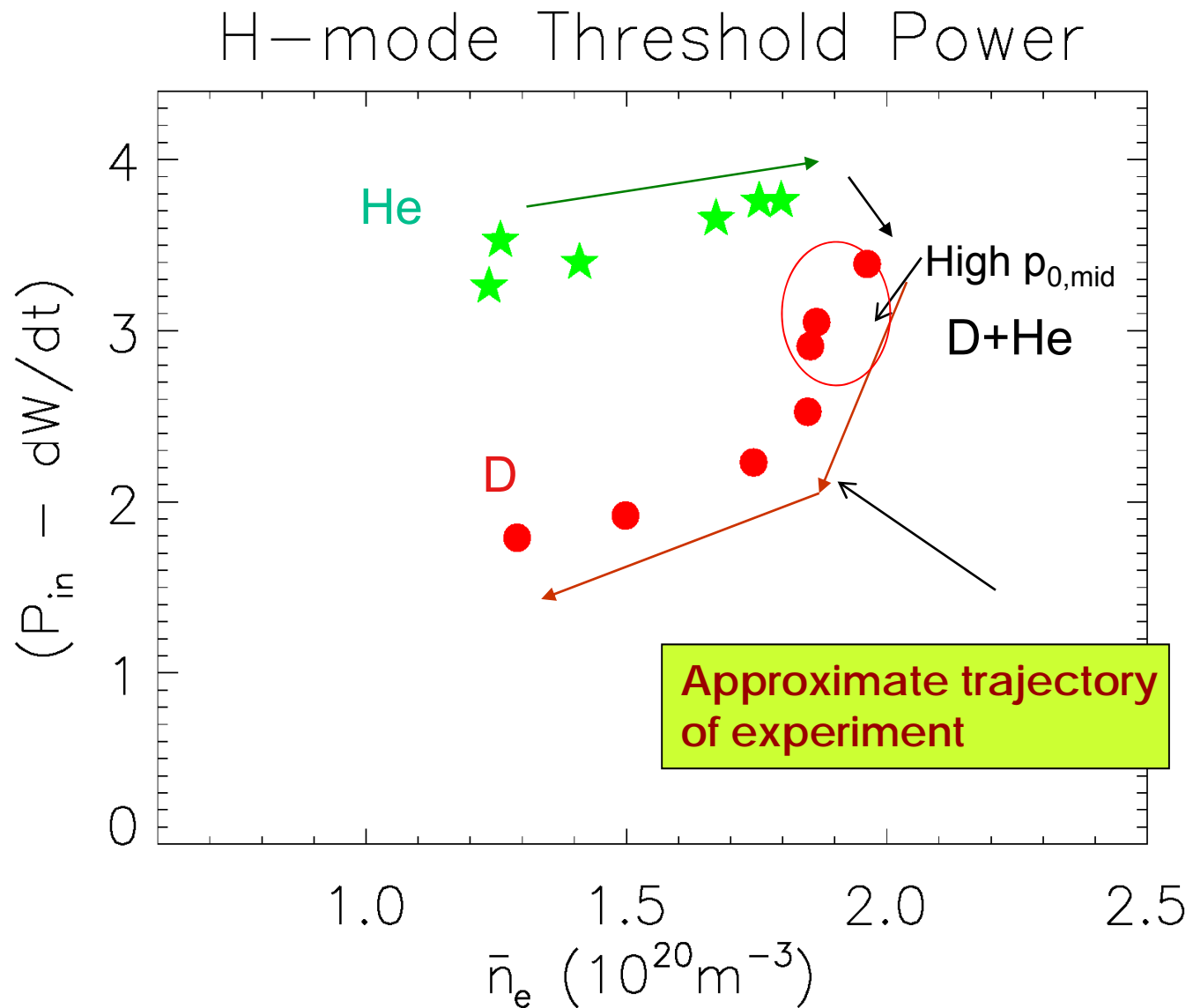
Results imply it is not necessary to attain the Connor-Hastie-Rosenbluth collisional limit to suppress runaways.



- Radiation measurements integrated over the disruption show a toroidal asymmetry between chords looking at and away from the gas jet; variable, up to 2.5.
 - A concern for ITER due to potential melting of Be.

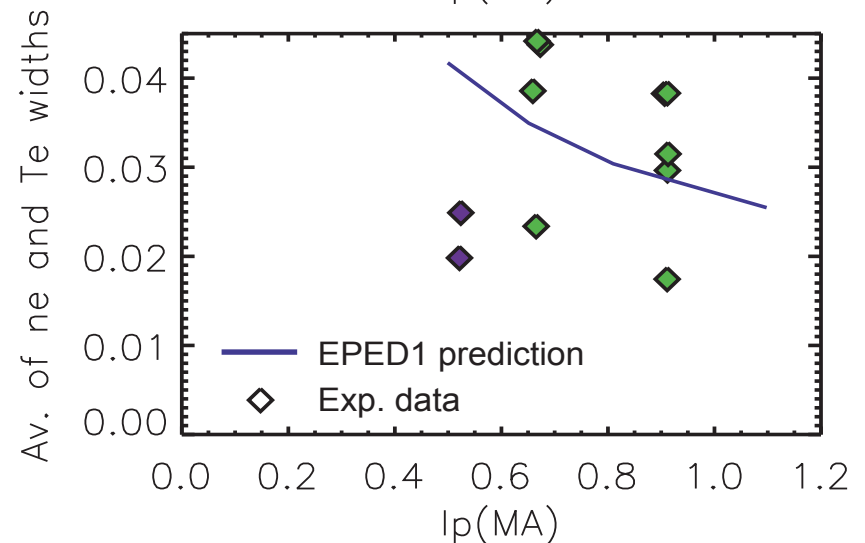
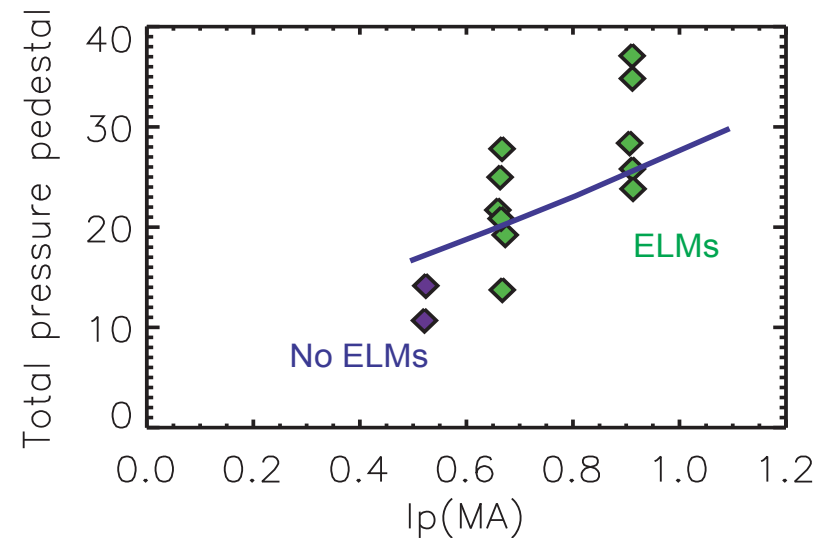
H-mode P_{thresh} Significantly Higher in He than D

- In ITER pre-nuclear phase, helium operation proposed to explore H-mode
- On C-Mod, see significant increase in threshold for He
- $P_{\text{th,He}}/P_{\text{th,D}} \sim 1.2 - 1.8$
- Comparisons with results from other facilities (JET, DIII-D, NSTX, MAST) ongoing through ITPA



EPED1* Simulations of H-mode Pedestal Showing ~Agreement with Experiment

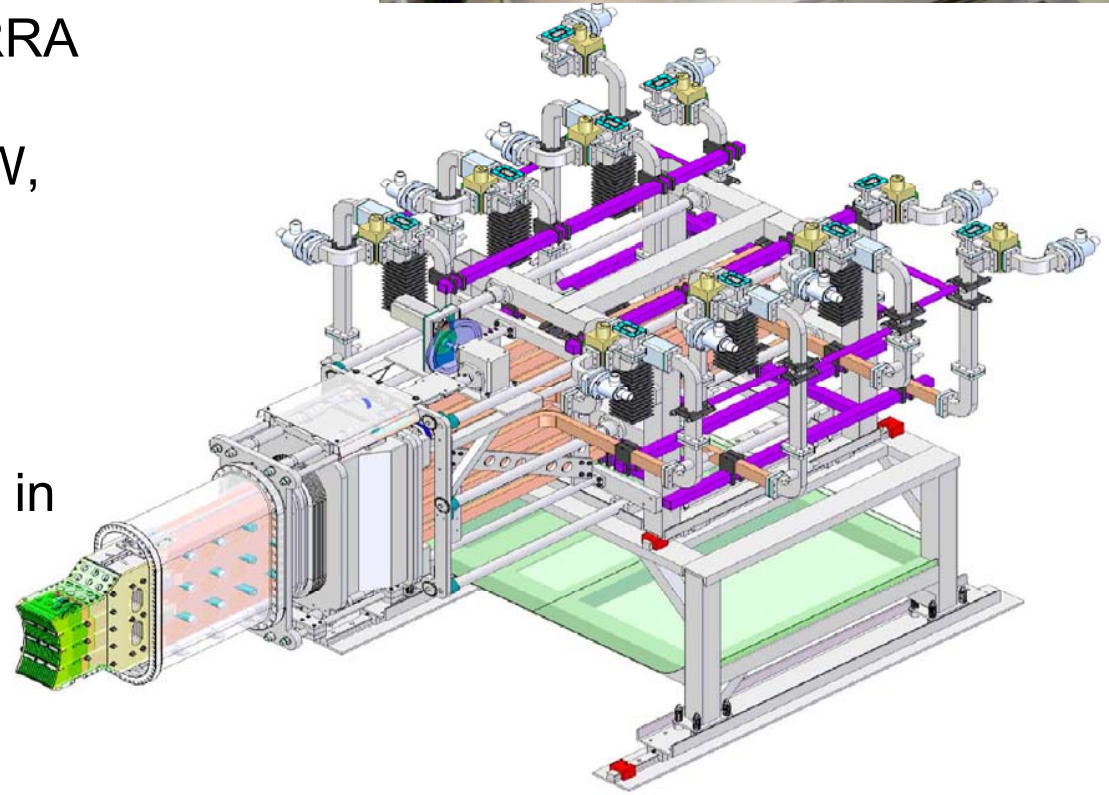
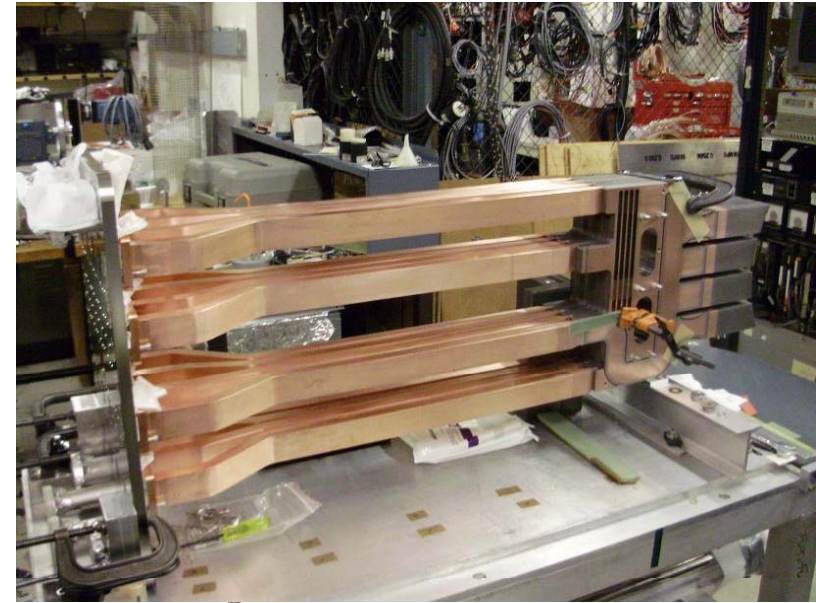
- Mix of ELMy, EDA H-modes
- Pedestal profiles obtained from time-averaging Thomson points during steady H-mode phases
- Initial EPED1 calculations (blue curves) made before the experiment predict maximum pedestal height and width (just prior to ELMs)
 - EPED1 predictions need to be revised based on actual beta and density obtained in experiment
- C-Mod contributions are expected to be especially useful in refining the next version(s) of EPED



*Collaboration with P. Snyder, GA

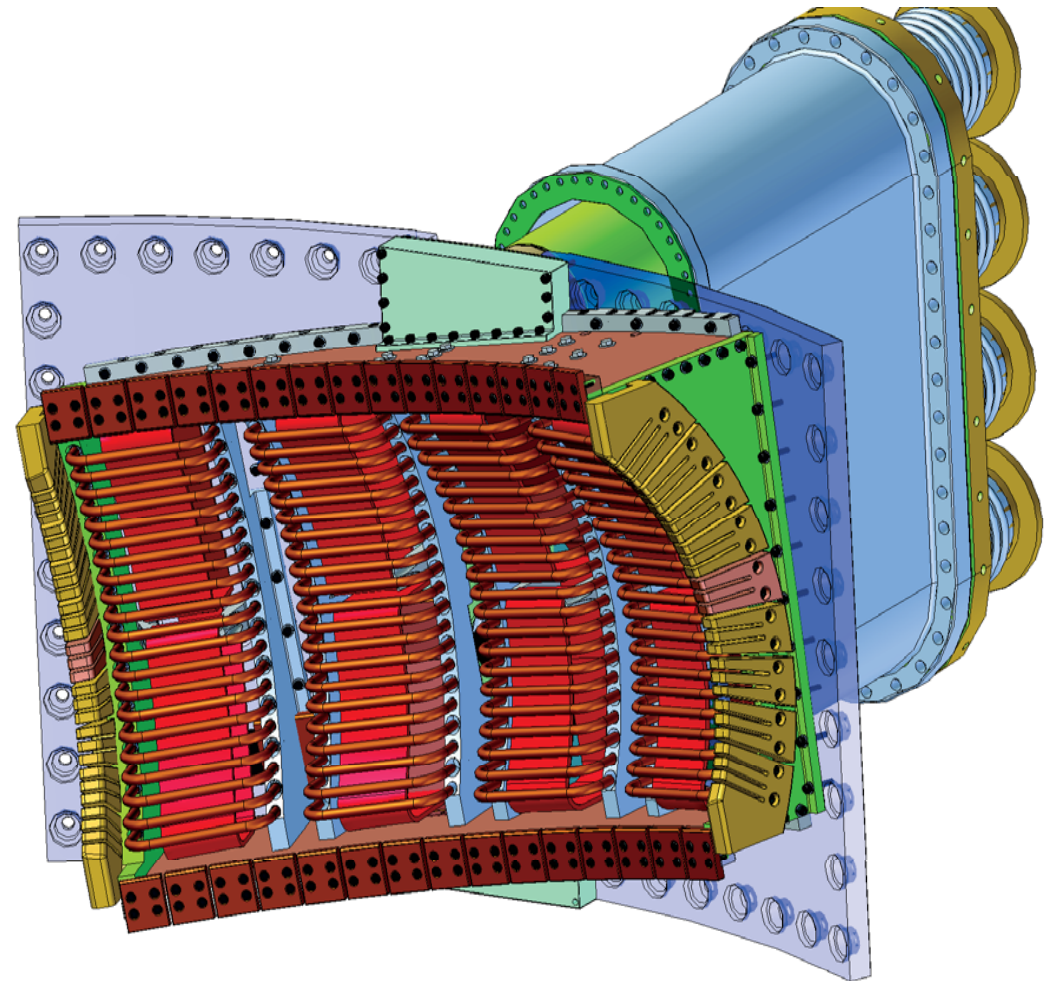
Advanced low-loss Lower Hybrid launcher

- Launcher upgrade
 - Low loss waveguide from the klystrons to the splitter
 - Expect up to 70% of source power to reach the vacuum interface
 - Uses novel load-tolerant 4-way splitters
- Upgrading source power (ARRA funded)
 - 7 new klystrons (≥ 250 kW, CW)
 - Adding 4th cart
 - Will bring total installed power to 4 MW
- New launcher being installed in December



Field-aligned ICRF antenna designed to reduce RF Sheath induced sputtering

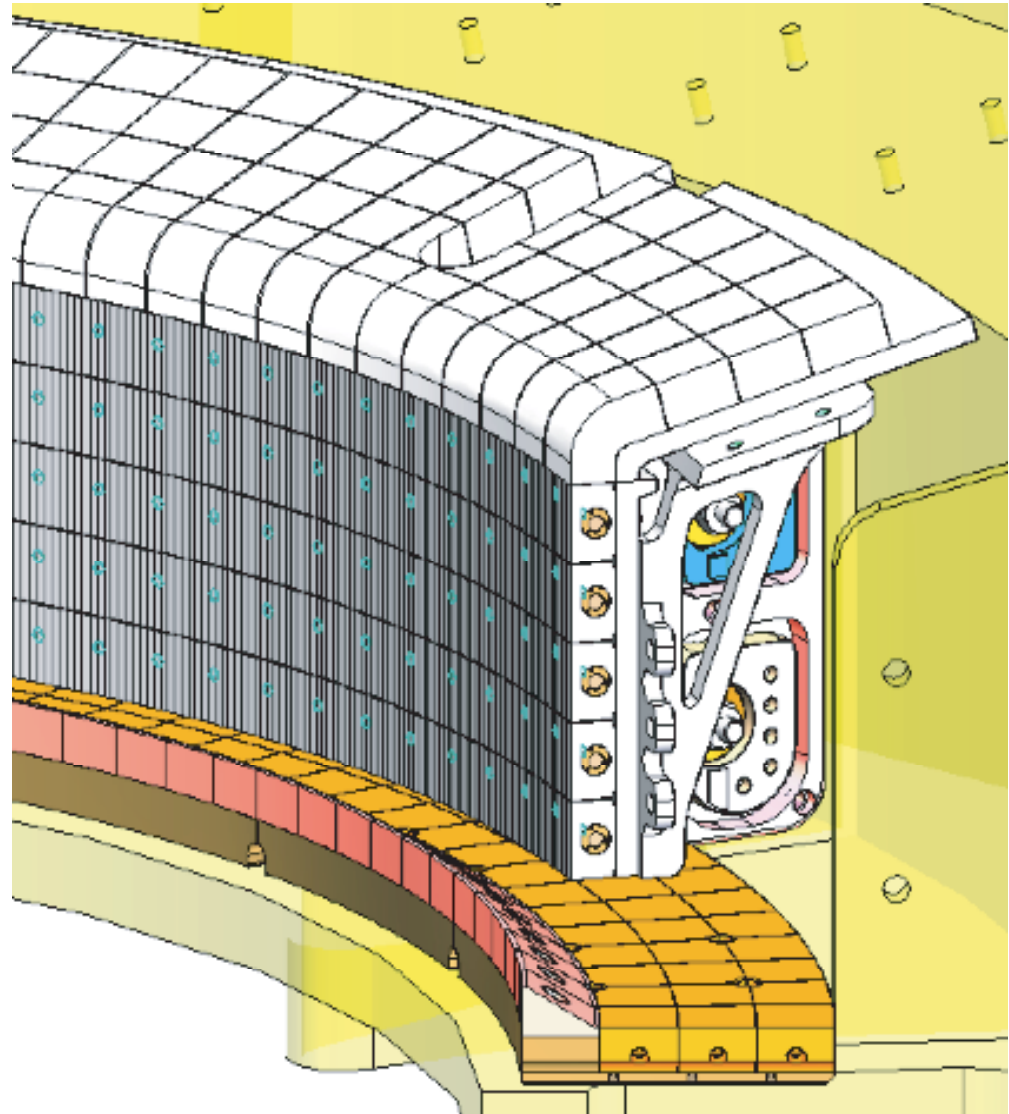
- ICRF induced sheaths implicated in high-Z impurity source production
- Rotating the antenna so that straps are perpendicular to local B
 - Modeling indicates substantial reduction of coupled E_{\parallel} (up to factor of 10 compared to vertical dipole).
- New antenna is in construction
 - Installation mid 2010



Advanced High Temperature Divertor

Motivation:

- Long Pulse - a new divertor design is required to extend the C-Mod pulse length (up to 5 seconds, 10 MW)
- High-Z studies and hydrogenic retention - an elevated operating temperature (up to ~600C) allows key exploration of:
 - Operational characteristics of tungsten
 - Fuel retention as a function of temperature.
- Both aspects drive the design toward a toroidally continuous, high-temperature divertor design.
- Joint effort with PPPL
 - Installation in FY2012



New Outer Divertor Conceptual Design

C-Mod FY2010 Campaign Well Underway



- Planning 18 total research weeks this fiscal year
 - Includes 5 weeks of incremental run-time funded through ARRA
 - Expect to complete the 5'th week tomorrow
- *New proposals and collaborations are always welcome!*

